





















Centrum výzkumu Řež s.r.o.

Post-irradiation inspections on TVSA-T fuel assemblies at Temelín NPP

VVER 2013

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12.11.2013



- CV Rez
- PIIP at Temelin NPP
- PIIP in 2013
- evaluation of FA changes with irradation
- FR growth
- corrosion situation





Background



CV Rez:

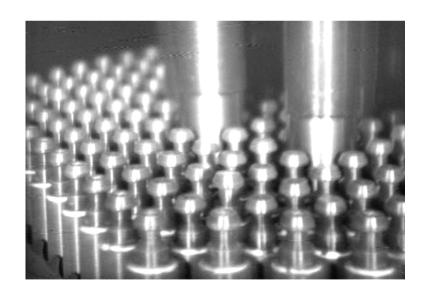
- Centrum výzkumu Řež s.r.o.
- member of UJV Group
- independent fuel inspector (in parallel with JSC TVEL) since 2011
- participant of PIIP since 2008







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PIIP at Temelin NPP



Post-Irradiation Inspection Program on TVSA-T includes:

- visual inspection of:
 - whole FA peripheral FRs, spacer grids, angles, upper nozzle, bottom nozzle
 - half-face
 - full-face
- measurements:
 - FA twist
 - FA bow
 - FA / FR growth
- UT inspection to identify the leaking FRs



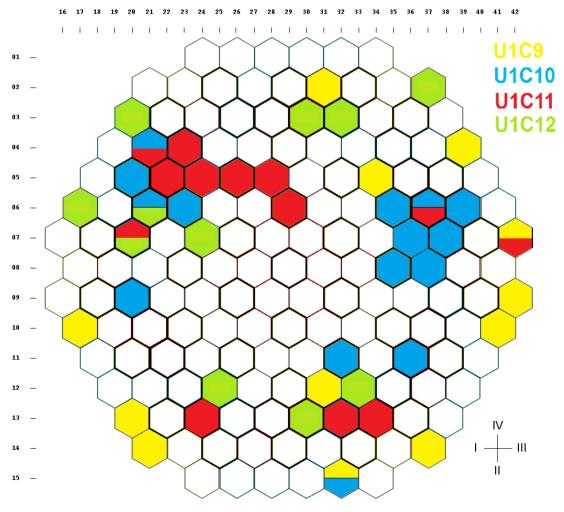


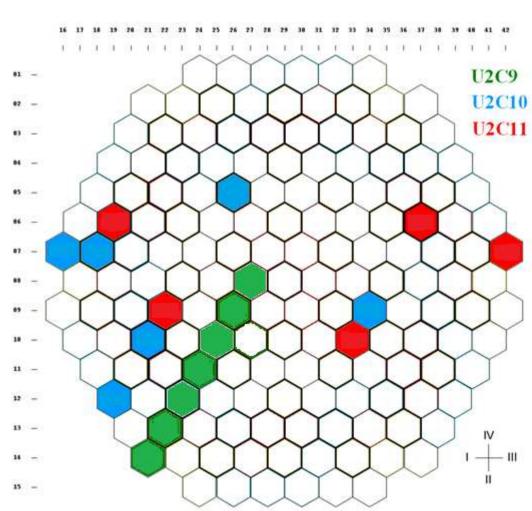


PIIP at Temelin NPP (2)



 selected FAs measured at U1 and U2 in 2011 – 2013 and 2014 (not all selected FAs yet)









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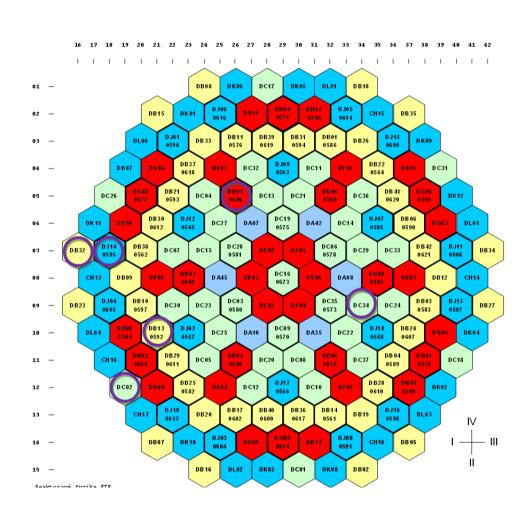




PIIP after U2C10



- Spring 2013
- after 2nd cycle with TVSA-T
- 1 one-year* and 5 two-year FAs
- BU: 15* 25 MWd/kgU
- no leakers
- measurement of 6 FAs:
 - twist ≤ 1°
 - bow \leq 5 mm (1 FA: 8 mm)
 - small difference of FA bow between U2C9 and U2C10
 - FA growth ≤ 2 mm
 - FR growth \rightarrow 3 8 mm

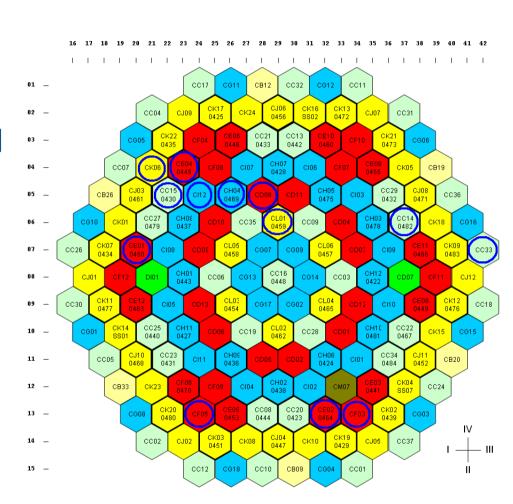




PIIP after U1C11



- July 2013
- after 3 cycles with TVSA-T
- BU: 16 37 MWd/kgU
- 9 three-year FAs, 2 two-year and 2 one-year FAs
- no leakers
- measurement of 13 FAs:
 - twist → immeasurable
 - bow \leq 4 mm (1 FA: 8 mm)
 - small difference of FA bow between U1C9, U1C10 and U1C11
 - FA growth ≤ 3.5 mm
 - FR growth \rightarrow 3 14 mm





Results: U2C10 and U1C11

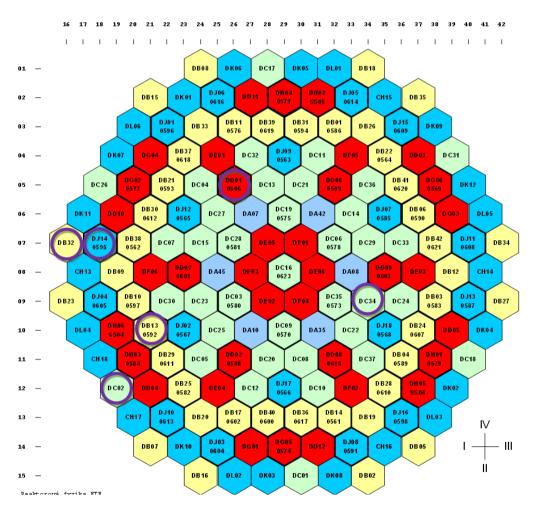


	U2C10	U1C11
FA age (years)	1 – 2	1 – 3
BU (MWd/kgU)	15 – 25	16 - 37
leakers	no	no
twist	~ 1°	~ 1°
bow	≤ 5 mm (1 FA: 8 mm)	≤ 8 mm
FA growth	≤ 2 mm	≤ 3.5 mm
FR growth	3 – 8 mm	3 – 14 mm

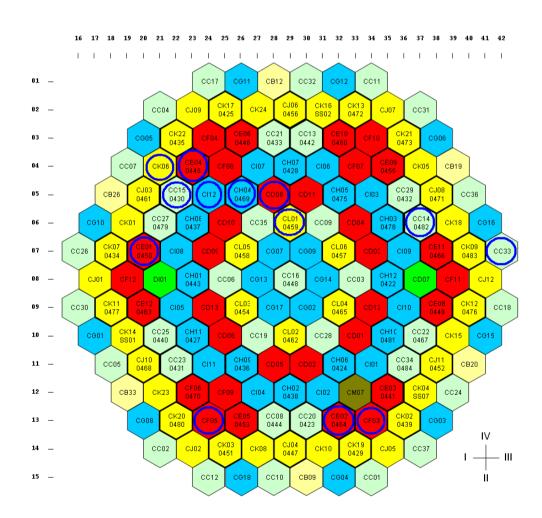
Results: U2C10 and U1C11 (2)



U2C10



U1C11







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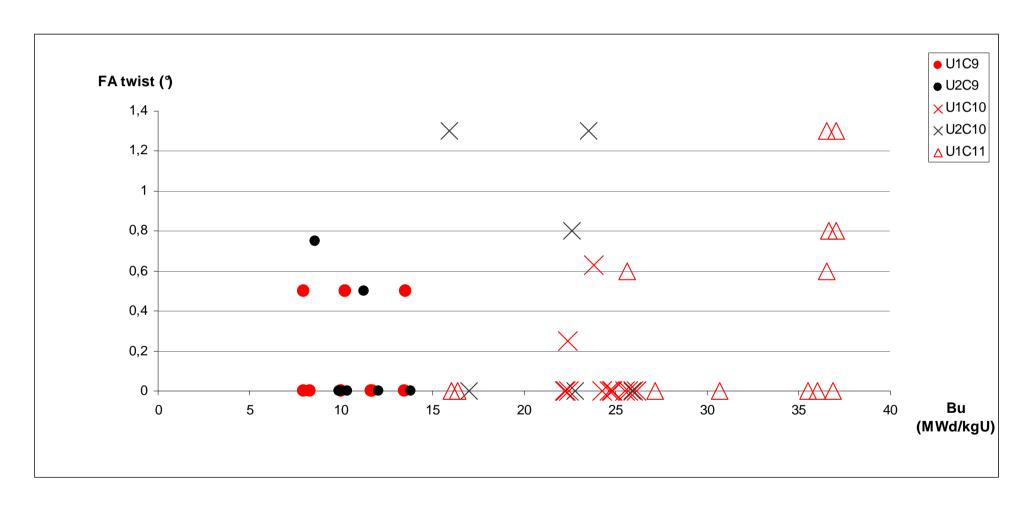




Evaluation of FA changes with irradiation



■ FA twist is negligible (up to 1.4°)

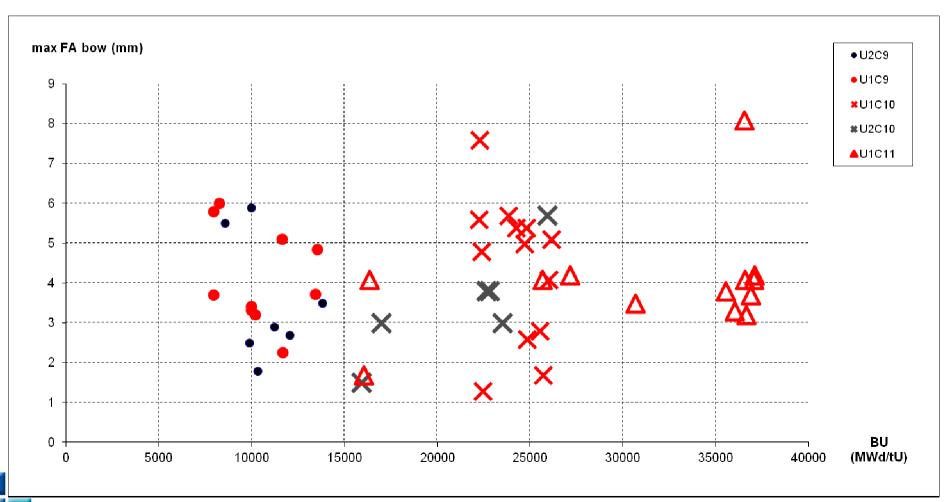




Evaluation of FA changes with irradiation (2)

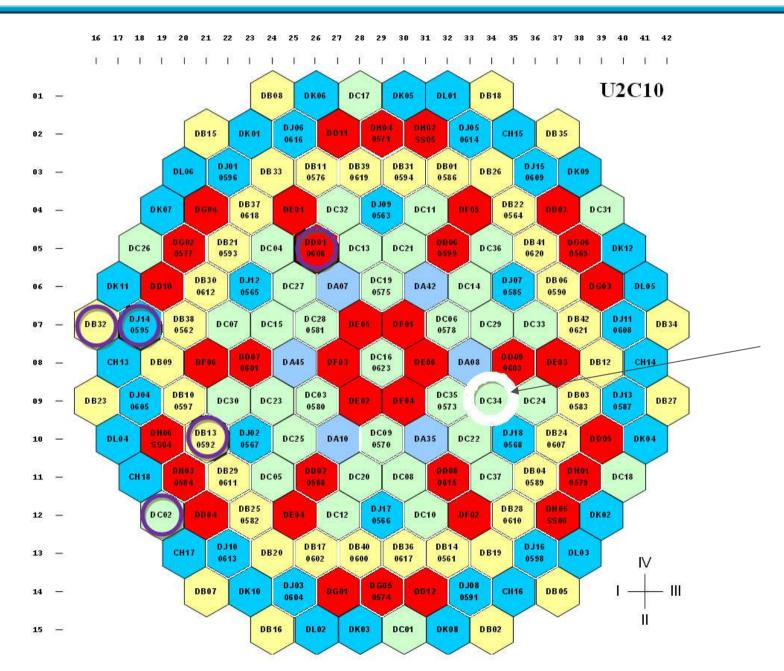


 FA bow meets the expectations and does not develop with burnup



U2C10



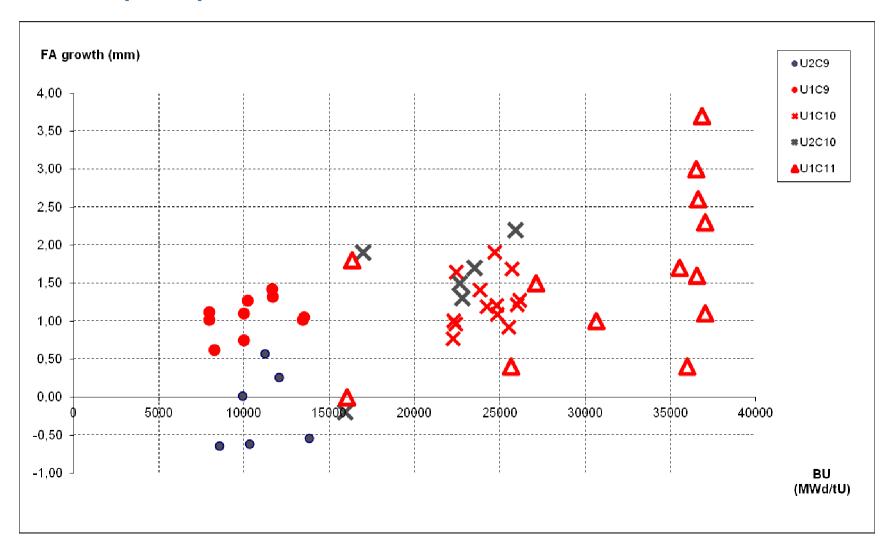




Evaluation of FA changes with irradiation (3)



 FA growth seems to be affected by the position of upper core internals (BOT)

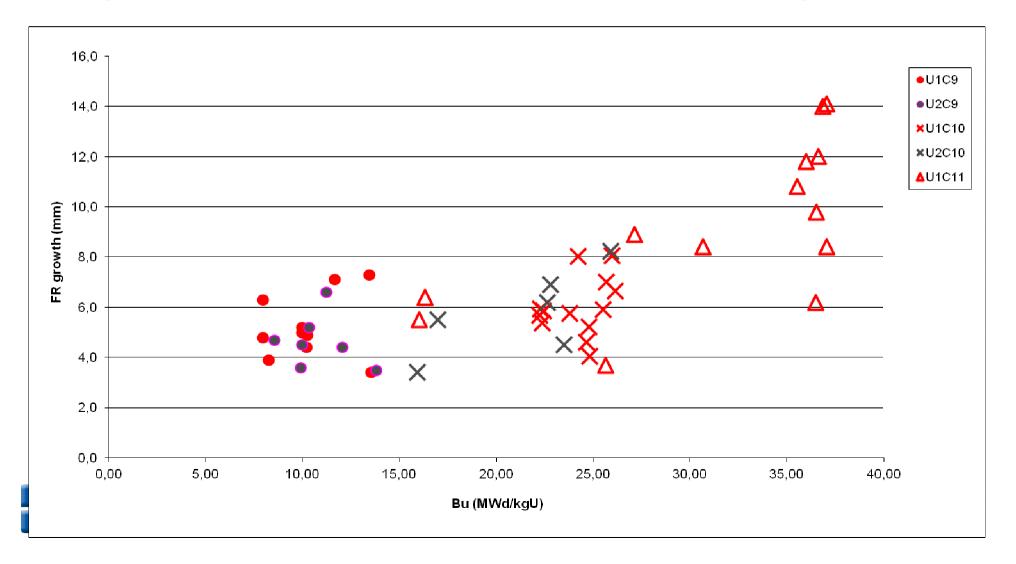




Evaluation of FA changes with irradiation (4)

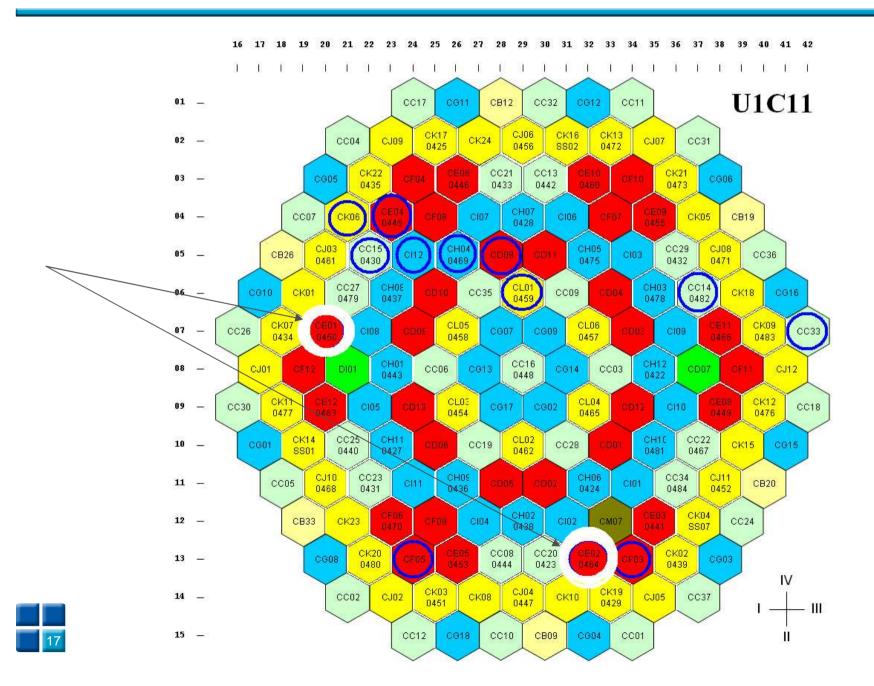


- FR growth is near the predicted value (0.1% / 10 MWd/kgU)
- Larger scatter after 3 cycles due to pellet-cladding contact



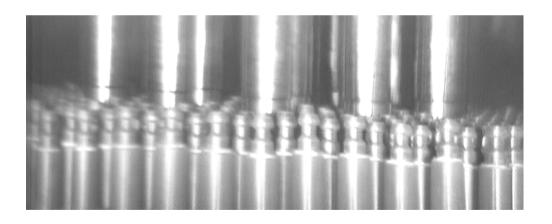
U1C11







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Fuel rod growth – tvel (UO₂ rods)



■ U1C9

■ 3 – 8 mm

U2C9

-3.5 - 7 mm



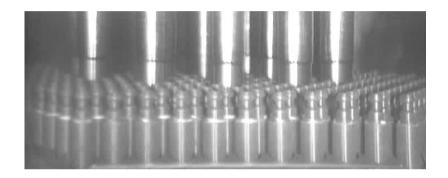
■ 4 – 8 mm

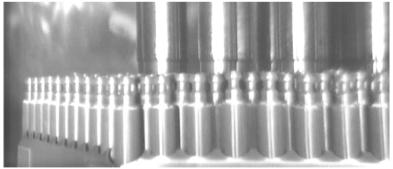


■ 3 – 8 mm

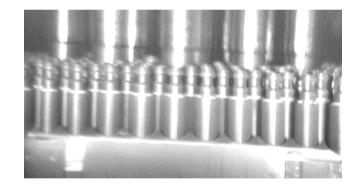
U1C11

• 3.5 - 14 mm









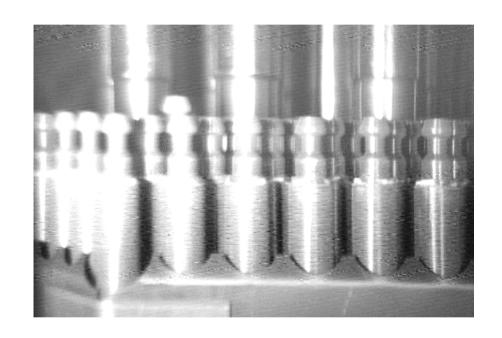




Fuel rod growth – tveg



- $UO_2 + Gd_2O_3 rod$
- small but noticeable difference between tvel and tveg growth observed already after 1st cycle at both units
- does not pose any safety or operational problem, but contradicted predictions
- observed only for some assemblies
- not common behavior of all tvegs





Tveg growth

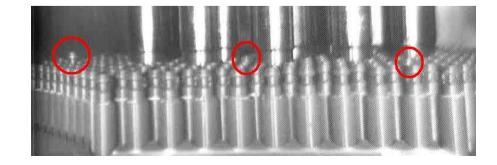


Discussion – considered but dismissed possible causes:

- different tveg lenght from the manufacture
- lower helium pressure under cladding and faster cladding creep
- different chemical composition or metalurgic properties of cladding tubes of certain batch
- lower end-plug release from the FA support grid
- pellet batch with higher content of hydrogen, or contamination of pellets or cladding tubes by organics leading to the hydriding of cladding

Most probable reason:

- Difference in pellet-clad contact moment between tvel and tveg
- According to calculations the the contact should occur slightly sooner in tveg, but not during the 1st cycle
- After 1st cycle the growth could be caused if the fuel column is not fully centered







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Corrosion situation

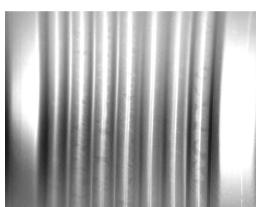


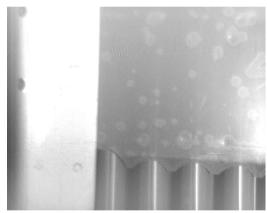
1 year



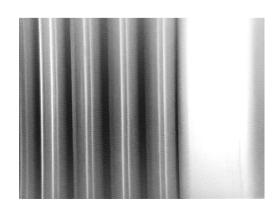


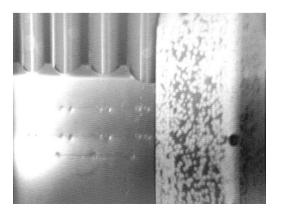
2 years





3 years







Conclusion



After 1, 2 and 3 cycles there are:

- no anomalies that would limit safe reactor operation
- no traces of handling damage found
- no defects of FA skeleton
- no significant FR growth
- mechanical stability
- FA behavior meets the predictions







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